

## Characterization of Radioactive Material at CERN

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## Introduction



 The material flow out of CERN accelerator facilities and experimental zones increases steadily: it generates from maintenance, repair, upgrade and decommissioning of existing installation

#### The particular conditions at CERN

- wide ranges of particle energies and physical conditions
- many different materials and material compounds
- various irradiation & cooling time scenarios

lead to a wide number of different and time dependent radio nuclide inventories

### Main question for

- safe handling
- transportation
- waste management

#### IS THIS MATERIAL RADIOACTIVE???

## PhD Thesis









#### 'Development of an In-Situ Radiological Classification Technique of Material from CERN's Accelerators and Experimental Facilities'

Supervised by Dr. Doris FORKEL-WIRTH, CERN Dr. Robert FROESCHL, CERN Prof. Rafael MACIAN-JUAN, TU Munich

## Projects of the last two years



- CERN material catalog / material guide lines / ActiWiz
- Installation and calibration of the total gamma chambers (RADOS RTM 661/440 und RTM 644)
- Measurement campaigns with RADOS RTM 661 & 644
- Material sampling, chemical and radio-chemical analysis
- Measurement campaigns with Canberra Falcon 5000 / ISOCS
- Material release measurements together with VKTA Dresden
- Characterization of LEP ventilation pipes
- Characterization of aluminum coils of the former LEP machine

# Material Characterization at CERN



### 3 Procedures

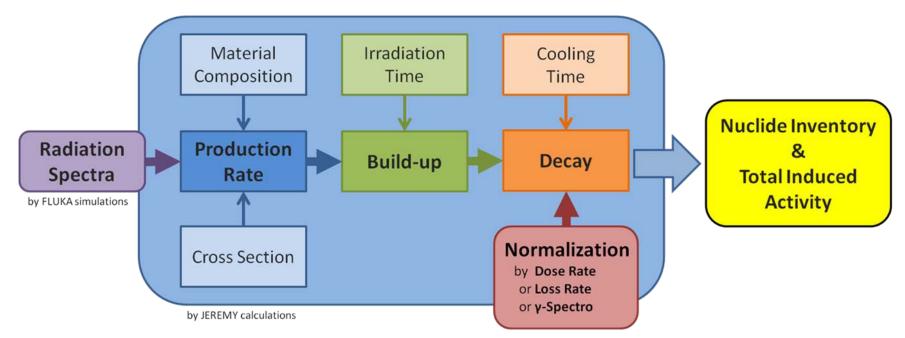
- "On site" operational radiation protection team
  - safe handling/transport
  - zoning concept
  - planning of future use/destination
- "No future use foreseen" radioactive waste team
  - safe treatment and optimized storage
  - classification in treatment classes
- Preparation for elimination radioactive waste team
  - identify elimination path
  - ensure compliance with elimination criteria



Material Characterization at CERN Characterization Methods

- Analytical calculations
- Monte Carlo simulations
- Alpha/beta counter (wipe tests)
- Gamma spectroscopy
- Total gamma chamber
- Dose rate measurements
- Chemical analysis
- Radio-chemical analysis

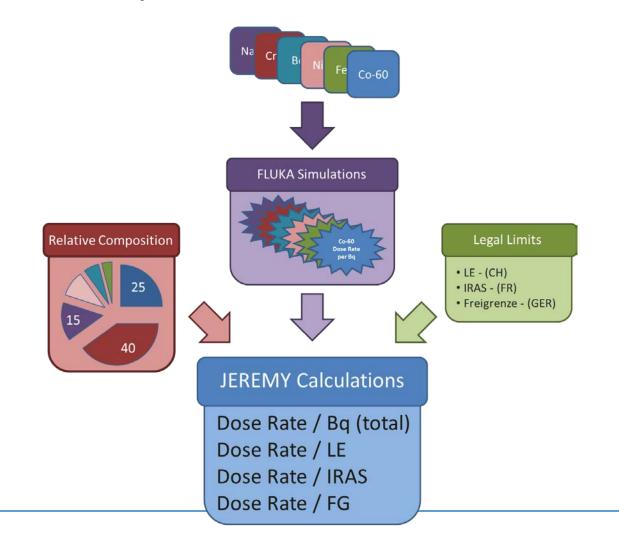
#### Radiation Spectra



### FLUKA parameters:

Scoring of particle fluence

- USRTRACK particle track length
- USRBDX particle boundary crossing



Dose Rate Activity Correlation

FLUKA parameters:

- **BEAM (ISOTOPE)**
- HI-PROBE (Z=27, A=60 -> Co60)
- **BEAMPOS** (Volume, Position) Scoring
- **USRBIN**

ose Rate [nanoSv/h]

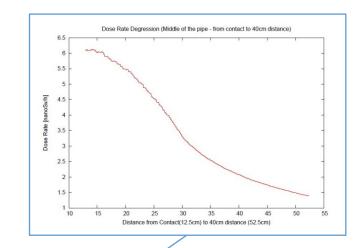
2

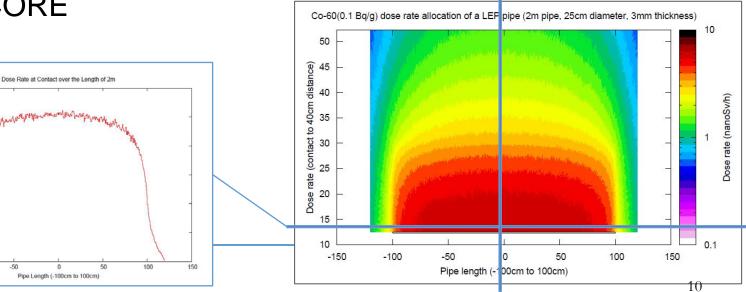
150

-100

-50

DCYSCORE





# How to integrate inhomogeneous activity distribution in FLUKA?



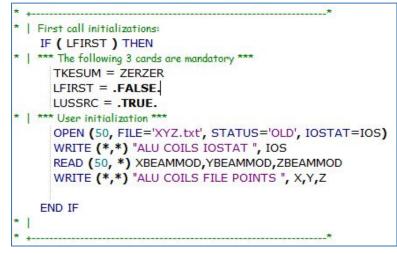


- Creation of a dose rate map (via dose rate meter)
- Calculation of fitted probability density function (pdf)
- "Random sampling" creation of pdf distributed random XYZ-coordinates (.txt file)
- FLUKA simulation with SOURCE routine
  - BEAM (photon, E=gamma line i.e. Na-22 = 1.275 MeV & 511 keV)
  - SOURCE.f (particle coordinates read out from the .txt file)
  - "call RACO" (random 3D direction)
  - USRBIN scoring of Dose Eq.

## FLUKA parameters: Changes in source.f

| * | Particle o | oordinates   |
|---|------------|--|
|   | IPCOL      | INTER = IPCOUNTER + 1  |
| - |            | OD(IPCOUNTER, 100) .EQ. 0) THEN  |
|   | WRI        | (50, *) XBEAMMOD,YBEAMMOD,ZBEAMMOD<br>TE (*,*) "ALU COILS FILE POINTS ", X,Y,Z |
|   | END I      |  |
|   | XFLK       | (NPFLKA) = XBEAMMOD  |
|   | YFLK       | (NPFLKA) = YBEAMMOD  |
|   | ZFLK       | (NPFLKA) = ZBEAMMOD  |

Each point is used 100 times



#### Initial read in of coordinates

| Cosines (tx,ty,tz)       |  |
|--------------------------|--|
| CALL RACO(TXX, TYY, TZZ) |  |
| TXFLK (NPFLKA) = TXX     |  |
| TYFLK (NPFLKA) = TYY     |  |
| TZFLK (NPFLKA) = TZZ     |  |
|                          |  |

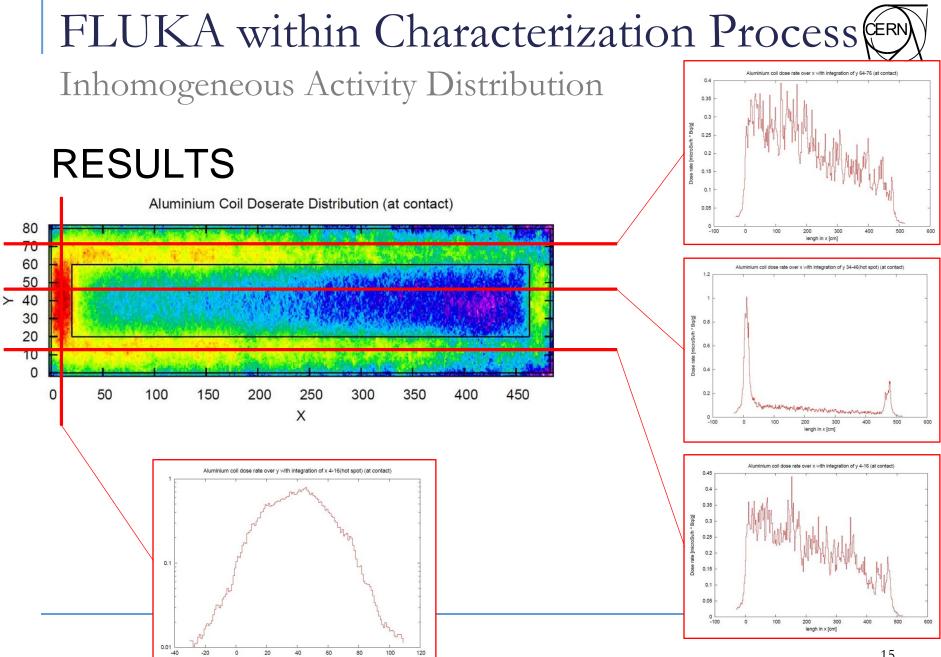
Random direction

### FLUKA parameters:

### Include .txt file in "rfluka"

#
DATAFILES="sigmapi.bin elasct.bin nuclear.bin fluodt.dat XYZ.txt"
XNLOANFIL="e6r1nds3.fyi jef2.fyi jendl3.fyi"

### Compile, create new executable & run



## Summary & Outlook



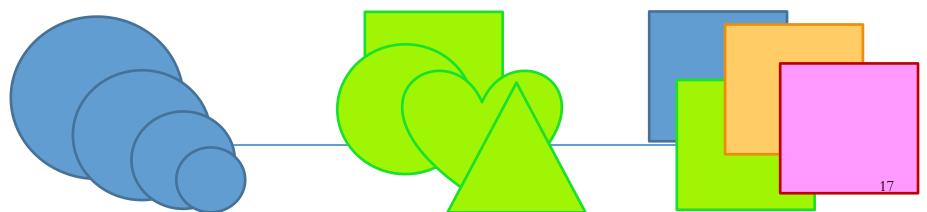
- FLUKA is a helpful tool for characterization
- Can be used in several procedures
- Simulation of inhomogeneous activity distribution possible

### Next steps

- Influence of geometry effects (with FLUKA)
- Combination of total gamma chamber and in-situ gamma spectroscopy

# FLUKA within characterization process self shielding effects

- The relative radionuclide inventory is determined
- Normalization i.e. per total gamma chamber
- Thus the attenuation length is material and energy dependent calibration measurements for almost all material/geometry combinations are required
- Not for all objects calibration measurements are feasible
- For homogeneous activity distribution within standard geometries and sizes the self shielding effects should be simulated and compared
- The goal is to gain have conservative self shielding factors for any geometry form, size and material combination



#### FLUKA within characterization process Total Gamma Chamber and in-situ Gamma Spectroscopy In-situ gamma spectroscopy focused on leading nuclides to select conservative nuclide inventory Material Composition worst case **Build-up** Radiation Production FAST 8 or Spectra Rate DECISION Decay by FLUKA simulations scenarios calculated **Cross Section Total gamma chamber** to normalize the nuclide inventory by JEREMY calculations

## Thank you for your attention!



### Comments, remarks and ideas are welcome!

Many thanks for their very positive contribution to:

#### Robert Froeschl Joao Saraiva James Chapman

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